

A Thrust for Integration of High-Performance Steady-State Burning-Plasma Behavior Relevant to Demo

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Fusion Research Themes (FESAC)

Theme A – Creating a High-Performance Steady-State Burning Plasma
(a heat source from magnetic fusion)

Theme B – Taming the Plasma Materials Interface
(interface between heat source and furnace wall, and extracting
plasma exhaust power)

Theme C – Harnessing the Power of Fusion
(extracting neutron power, breeding tritium, remote handling,
safety/environment)

- These themes follow a systems or process based approach, sometimes called Holistic approach to the R&D of complex systems - eg space craft.
- They form a natural overlapping sequence

Theme A

Theme B

Theme C

Key Questions related to the Physics of a Demo Plasma

- A Demo plasma with high fusion gain, high neutron wall loading, high bootstrap fraction for efficient steady state and high power density plasma exhaust is a highly integrated plasma system. Some questions:
 - Is good confinement compatible with P_α defined profiles?
 - Does transport depend on non-linearly on the pressure profile?
 - Is high beta compatible with P_α defined profiles?
 - Does the plasma evolve to a stable self-organized state?
 - Will alpha heating drive a self-heating sawtooth?
 - Can the plasma be sustained and controlled with low power?
 - What are the optimum temperature and density regimes for simultaneous high Q, efficient CD and long life divertor operation?
 - Many more.....

- Can we quantify the gaps between today, ITER and a Demo?

High-Performance Steady-State Burning-Plasma Issues

High Fusion Gain - attain good confinement with profiles defined by alpha heating ($P_\alpha/P_{\text{ext}} = Q/5$), possible non-linear dependence of transport on gradients, coupled to edge plasma by pedestal, optimum temperature for fusion ~ 15 keV and high density but efficient current drive favors higher $T \sim 30$ keV and lower density.

Sustainment (100% NI) - produce large bootstrap current with pressure profiles defined by alpha heating and residual current driven efficiently by low power $P_{\text{cd}} \leq 5P_\alpha/Q$.

High Fusion Power Density ($\beta^2 B^4 \langle \sigma v \rangle / T^2$) - to provide high neutron wall loading. Can near optimum β be attained for alpha-defined profiles?

Plasma Control ($P_{\text{cd}} + P_{\text{cont}} = 5P_\alpha/Q$) - maintain plasma control (esp. disruptions) with low power typically $< 0.15P_\alpha$. Will a burning plasma evolve to a self-organized state with good confinement, high bootstrap and high β ?

Exhaust Power Density - can high exhaust power densities be handled while maintaining edge plasma for high Q and efficient CD with long PFC lifetime?

Self-Conditioned PFCs - will the PFCs self-condition that is consistent with high Q and β , and long PFC lifetime?

High-Performance Steady-State Burning-Plasma Metrics and Gaps

Table I. Individual Issue (Metric)	Today* ($>10\tau_E$)	ITER	ARIES-I	ARIES-AT	<Gap> IT to AR
Fusion Gain (Q)	< 0.2	5	20	50	7
Self-heating (%)	4	50	80	91	1.7
Sustainment (100% NI)** (P_{cd}/P_α)	>25	1	0.25	0.1	6
Current Drive fraction ($1-f_{bs}$) (%)	~ 30	~ 50	32	9	2.5
Neutron Wall Loading (MWm^{-2})	0.1	0.5	2.5	3.3	6
Plasma Pressure (atm)	1.6	2.5	10	10	4
Fusion Power density (MWm^{-3})	0.3	0.5	4	4.7	8
Plasma Control* (P_{cont}/P_α)	>25	1	0.25	0.1	6
Exhaust Power Density (P_{heat}/A_{ps}) (MWm^{-2})	0.85	0.2	1	1	5
Self-Condition PFCs & FW $f(t_{pulse}, T, \phi,$	No	?	Yes	Yes	?

* Not all simultaneous

** Current Drive Power + Plasma Control Power = $5 P_\alpha/Q$

Assumes ITER will be upgraded with addition of Lower Hybrid current drive for Scenario 4.

- ARIES-I And ARIES-AT span the range of a possible DEMO.
- Individual gaps between ITER (scenario 4) and ARIES range between 1.7 and 10

Description of integration issues

High-Performance Steady-State Burning-Plasma Integration Issue Gaps (an example)

Integrate Fusion Gain, Sustainment and Exhaust Power Density

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- The individual gaps are taken to be independent, therefore the Integration Gap is the product of individual gaps.
- **The Integration Gap for Fusion Gain, Sustainment and Exhaust Power density is ≈ 200**

A Thrust for Integration of High-Performance Steady-State Burning-Plasma Behavior Relevant to Demo

Key Objectives of Thrust

- Determine and understand conditions for attaining a Demo-relevant burning plasma.
- Determine and understand conditions for sustaining and controlling a Demo-relevant plasma that is dominately self-heated, with dominately self-driven currents and dominately self-conditioned PFCs.
- Test and Refine Predictive Modeling on a Demo relevant plasma.
- Together with ITER, and other Thrusts provide the knowledge basis for the design of a tokamak based Demo.

Strategy for Integrating Demo Relevant Plasma Issues

- Aggressively exploit simulation on existing DD facilities and computer models
 - target specific objectives/tasks with SC action teams
 - exploit Asian superconducting facilities
 - simulate burning plasma phenomena to the extent possible
- Begin a study of the Fusion Plasma Integration Facility that would address the integration issues of a Demo-relevant High-Performance Steady-State Burning plasma and serve as a D-T satellite tokamak for ITER.
 - refine key objectives and research requirements
 - define general characteristics of possible facilities (iterate with above)
 - since the cost will be significant, start with a plan that has a sequence of upgrades that spreads the cost and allows success to bootstrap funding for the next stage or objective.
 - begin the pre-conceptual design of a facility(s) within a year to assess technical feasibility and cost range.

Note: Not building a major Burning Plasma facility is very expensive

Since 1997 US MFE has spent \$3.4B (\approx \$4B in FY08 \$)

Since 1989 US MFE has spent \$6B (\approx \$7.5B in FY08 \$)